evaluated the accidental release of radioactive materials. The intakes and predicted doses for three of the radiological accident scenarios were small with negligible associated health impacts. The fourth radiological accident, rupture of a UF_6 cylinder (liquid), produced a dose of 3.9 rem at the nearest resident. While the potential consequences of such an event would be severe, the likelihood of such an event is low because of design and procedural controls. The fifth accident analyzed, the release of gaseous ammonia, would be expected to produce noticeable, but non-life-threatening effects both onsite and offsite. Given the low likelihood for these accidents, it is concluded that the proposed license renewal will not have a significant impact on the general population.

Alternatives to the Proposed Action

Alternatives to the proposed action include denial of Allied's renewal application. Not granting a license renewal for the facility would cause Allied to cease production of UF_6 at this site. The only benefits to be gained by nonrenewal would be the cessation of the minor environmental impact from operation of the facility. Because Allied's site is the only operating facility to convert uranium ore to UF_6, denial of a license for Allied would result in the transfer of the activity and associated environmental impact to an alternative site.

Agencies and Persons Consulted

The staff utilized the application dated July 11, 1994, and additional information dated September 6, and November 16, 1994. Discussions were held with the Agreement States of Illinois and Kentucky. The Region III inspectors and Allied representatives were also consulted in preparing this document.

Conclusion

The staff concludes that the environmental impacts associated with the proposed license renewal for continued operation of Allied's Metropolis facility are expected to be insignificant.

Finding of No Significant Impact

The NRC has prepared an Environmental Assessment related to the renewal of Source Material License SUB–526. On the basis of the assessment, NRC has concluded that environmental impacts that would be created by the proposed licensing action would not be significant and do not warrant the preparation of an Environmental Impact Statement. Accordingly, it has been determined that a Finding of No Significant Impact is appropriate.

The Environmental Assessment and the documents related to this proposed action are available for public inspection and copying at NRC's Public Document Room at the Gelman Building, 2120 L Street NW, Washington, DC.

Opportunity for a Hearing

Any person whose interest may be affected by the issuance of this renewal may file a request for a hearing. Any request for hearing must be filed with the Office of the Secretary, U.S. Nuclear Regulatory Commission, Washington, DC 20555, within 30 days of the publication of this notice in the **Federal Register**; be served on the NRC staff (Executive Director for Operations, One White Flint North, 11555 Rockville Pike, Rockville, MD 20852), and on the licensee (AlliedSignal, Inc., Route 45 North, P.O. Box 430, Metropolis, IL 62960); and must comply with the requirements for requesting a hearing set forth in the Commission's regulation, 10 CFR Part 2, Subpart L, ''Informal Hearing Procedures for Adjudications in Materials Licensing Proceedings.''

These requirements, which the requestor must address in detail, are:

1. The interest of the requestor in the proceeding;

2. How that interest may be affected by the results of the proceeding, including the reasons why the requestor should be permitted a hearing;

3. The requestor's areas of concern about the licensing activity that is the subject matter of the proceeding; and

4. The circumstances establishing that the request for hearing is timely, that is, filed within 30 days of the date of this notice.

In addressing how the requestor's interest may be affected by the proceeding, the request should describe the nature of the requestor's right under the Atomic Energy Act of 1954, as amended, to be made a party to the proceeding; the nature and extent of the requestor's property, financial, or other (i.e., health, safety) interest in the proceeding; and the possible effect of any order that may be entered in the proceeding upon the requester's interest.

Dated at Rockville, Maryland, this 11th day of May 1995.

For the Nuclear Regulatory Commission.

Robert C. Pierson,

Chief, Licensing Branch, Division of Fuel Cycle Safety and Safeguards, NMSS.

[FR Doc. 95–12103 Filed 5–16–95; 8:45 am]

BILLING CODE 7590–01–P

[Docket No. 72–10 (50–282/306)

Northern States Power Co. Prairie Island Nuclear Generating Plant Independent Spent Fuel Storage Installation; Exemption

I

Northern States Power Company (NSP or the licensee) holds materials license (SNM–2506) for receipt and storage of spent fuel from its Prairie Island Nuclear Generating Plant at an independent spent fuel storage installation (ISFSI) located on the Prairie Island Nuclear Generating Plant site. This facility is located at the licensee's site in Goodhue County, Minnesota.

II

Pursuant to 10 CFR 72.7, the Nuclear Regulatory Commission (NRC) may grant exemptions from the requirements of the regulations in 10 CFR Part 72 as it determines are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest.

Section 72.82(e) of 10 CFR Part 72 requires each licensee to provide a report of preoperational test acceptance criteria and test results to the appropriate NRC Regional Office with a copy to the Director, Office of Nuclear Material Safety and Safeguards, at least 30 days prior to receipt of spent fuel or high level radioactive waste for storage in an ISFSI. The purpose of the 30-day period is to allow the NRC an opportunity to review test results prior to initial operation of the ISFSI.

III

By letter dated January 4, 1995, the licensee requested a schedular exemption pursuant to 10 CFR 72.7 from the requirement of 10 CFR 72.82(e). The licensee committed to submit its report no less than 3 days prior to receipt of spent fuel at its ISFSI.

In July 1993, NSP suspended cask fabrication and site construction activities until the Minnesota State Legislature authorized the ISFSI on May 10, 1994. After authorization, NSP resumed the ISFSI construction and the facility was completed in November 1994. The fist cask was received on January 26, 1995.

The NRC conducted an inspection of the quality assurance records related to the manufacture of the cask at vendor sites, and on October 11, 1994, and January 25, 1995, issued Inspection Reports Nos. 72–0010/94–210 and 72–0010/94–212, respectively, to NSP. On February 23, 1995, and March 8, 1995, NSP responded to the Notice of

Violation in the inspection report and provided additional information. In a letter dated March 21, 1995, NRC found NSP's corrective actions acceptable. Since receipt of the first cask on site, NRC has observed selected portions of the preoperational testing activities and has reviewed associated test procedures and results. In addition, during the weeks of April 17, and April 24, 1995, the NRC conducted a special team inspection of cask fabrication records and preoperational test results at the Prairie Island Nuclear Generating Plant. On April 28, 1995, NRC held an inspection exit meeting, which was open to public attendance, in Red Wing, Minnesota, to discuss its inspection findings and conclusions. NSP submitted the report of preoperational test acceptance criteria and test results required by 10 CFR 72.83(e) to the NRC on April 20, 1995. At the inspection exit meeting held on April 28, 1995, the following five outstanding issues and their resolution were:

Issue (1): Fabrication of the temperature and pressure monitoring equipment was not complete.

Resolution: NRC Resident Inspectors observed completion of installation of fabricated equipment.

Issue (2): NRC review of the unloading procedure was not complete.

Resolution: NRC Resident Inspectors have completed their review and all identified concerns have been acceptably resolved.

Issue (3): NRC review of licensee disposition of weld discrepancies was not complete.

Resolution: NRC staff have completed their review and have accepted the licensee's dispositions.

Issue (4): Resolution of cask hydrostatic testing requirements was not complete.

Resolution: NRC staff have resolved the cask hydrostatic testing requirements. In addition, the licensee performed a 10 CFR 72.48 evaluation to revise the Safety Analysis Report. The Resident Inspectors have reviewed the evaluation and found it acceptable.

Issue (5): NRC review of adequate spent fuel retrievability was not complete.

Resolution: The licensee provided information regarding retrievability in a letter dated May 3, 1995. In a letter to NSP dated May 5, 1995, NRC found NSP's rationale acceptable.

Based on the resolutions described above, the staff has completed its review and is granting the exemption.

An exemption to the requirement of 10 CFR 72.82(e) for a 30-day waiting period would allow NSP to start loading the first cask before May 20, 1995, the end of the 30-day period.

IV

As previously described in the foregoing discussion, and based on its oversight and inspection of NSP's ISFSI preoperational testing activities, the NRC finds that NSP has satisfactorily addressed all of the outstanding safety issues associated with cask loading, handling, and storage. The results of the NRC activities described above confirm there is adequate assurance that the cask can perform its intended safety functions and that NSP has the necessary equipment and procedures in place to safely conduct spent fuel cask handling activities.

Accordingly, the NRC has determined in accordance with 10 CFR 72.7 that this exemption is authorized by law, will not endanger life or property or the common defense and security, and is otherwise in the public interest. Therefore, the NRC hereby grants the licensee an exemption from the 30-day waiting period required by 10 CFR 72.82(e) as requested by the licensee's letter of January 4, 1995.

The documents related to this proposed action are available for public inspection and for copying (for a fee) at the NRC Public Document Room, 2120 L Street, NW, Washington, DC 20555, and at the Local Public Document Room located in the Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401.

Pursuant to 10 CFR 51.32, the NRC has determined that granting this exemption will have no significant impact on the quality of the human environment (60 FR 13477).

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 11 day of May 1995.

Donald A. Cool,

Director, Division of Industrial and Medical Nuclear Safety, Office of Nuclear Material Safety and Safeguards.

[FR Doc. 95–12100 Filed 5–16–95; 8:45 am]

BILLING CODE 7590–01–M

Elimination of Low-Level Radioactive Waste Topical Report Review Program

AGENCY: Nuclear Regulatory Commission.

ACTION: Notice of elimination of program.

SUMMARY: This notice is in reference to low-level radioactive waste (LLW) topical reports (TRs) submitted in support of the implementation of 10 CFR Part 61, or compatible Agreement State regulations. The U.S. Nuclear Regulatory Commission's Division of Waste Management (DWM) is currently responsible for the Federal review of these TRs. However, due to higher priorities and limited staff availability, DWM has decided to terminate its LLW TR review program.

ADDRESSES: Documents referred to in this notice may be examined at the Commission's Public Document Room, 1717 H Street, NW., Washington, DC. Copies of technical positions and topical report review procedures may be obtained from the Division of Waste Management, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555, or by calling the contact listed below.

FOR FURTHER INFORMATION CONTACT: Robert J. Lewis, Engineering and Geosciences Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC 20555; telephone (301) 415–6680.

SUPPLEMENTARY INFORMATION: To help ensure a cost effective and orderly implementation of 10 CFR Part 61, and since many licensees utilize similar services from the same firm, the U.S. Nuclear Regulatory Commission issued a **Federal Register** notice (48 FR 40512) encouraging these firms to submit, for NRC review, generic topical reports on these services.

To meet 10 CFR 20.2006(d), 10 CFR 61.55, and 10 CFR 61.56, radioactive waste generators and disposal operators must demonstrate compliance with the waste classification and waste form requirements. One acceptable approach to satisfying these regulations is to reference previously reviewed and approved TRs. A TR is a document submitted by an industry organization (i.e., a vendor) for review by NRC or an Agreement State, outside of specific licensing action. LLW TRs typically include reports on qualification of high-integrity containers, solidification procedures, or computer codes designed to classify waste.

The ultimate acceptability of a particular waste is subject to the disposal restrictions and requirements specified by the waste disposal facility operators and governing Agreement State regulatory agencies. NRC approved LLW TRs are often accepted by a State as an acceptable means of demonstrating compliance with the State equivalent regulations to 10 CFR Part 61. However, due to higher priorities and limited staff availability, DWM has decided to terminate the LLW TR review program. The number of LLW